

July 15, 1985  
NRC/THI-85-051

MEMORANDUM FOR: Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
Bernard J. Snyder, Program Director  
THI Program Office

FROM: William D. Travers, Deputy Program Director  
THI Program Office

SUBJECT: NRC THI PROGRAM OFFICE WEEKLY STATUS REPORT FOR  
JULY 8, 1985 - JULY 14, 1985

1. PLANT STATUS

- The facility remains in long term cold shutdown with the Reactor Coolant System (RCS) vented to the reactor building atmosphere and the reactor vessel head and plenum assembly removed.
- The reactor vessel plenum has been removed from the reactor vessel and placed on its storage stand in the deep end of the fuel transfer canal. A dam has been installed between the deep and shallow ends of the fuel transfer canal. The deep end is filled with water to a depth of about 20 feet (about 5 feet above the top of the plenum).
- The modified internals indexing fixture is installed on the reactor vessel flange and is flooded to elevation 327 feet 6 inches (15½ feet above the top of the core region).
- Calculated reactor decay heat is less than 12 kilowatts.
- RCS cooling is by natural heat loss to the reactor building ambient atmosphere. Incore thermocouple readings range from 72°F to 93°F with an average of 82°F. Average cold leg temperature is 66°F.
- The average reactor building temperature is 63°F. The reactor building airborne activity is 1.4 E-7 uCi/cc tritium, and 7.6 E-10 uCi/cc particulate, predominantly cesium 137.

2. WASTE MANAGEMENT

- The Submerged Demineralizer System (SDS) commenced processing batch 122 consisting of 13,358 gallons from the "B" reactor coolant bleed tank. EPICOR II commenced processing batch 258 consisting of 7,765 gallons from SDS tank 1B.
- Total volume processed through SDS to date is 2,912,194 gallons, and the total volume processed through EPICOR II is 2,488,539 gallons.

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3. DOSE REDUCTION/DECONTAMINATION ACTIVITIES

- Shielding of the lower portion of the reactor building air coolers has been completed. Shielding of the upper (motor) portion is being re-evaluated on the basis of ALARA. Dose rate reduction factors of 2 to 3 were realized from shielding of the lower portions.
- Average general area radiation dose rate is 30 mrem per hour on the 347' level of the reactor building and is 160 mrem per hour on the 305' level of the reactor building.

4. ENVIRONMENTAL MONITORING

- EPA sample analysis results show THI site liquid effluents to be in accordance with regulatory limits, NRC requirements, and the City of Lancaster Agreement.
- THI water samples taken by the US Environmental Protection Agency at the plant discharge to the river consisted of seven daily composite samples taken from June 22, to June 29, 1985. Gamma scans detected no reactor related radioactivity.
- The Lancaster water sample taken at the water works intake and analyzed by the US Environmental Protection Agency consisted of a seven day composite sample taken from June 23, to June 29, 1985. A gamma scan detected no reactor related radioactivity.
- The NRC outdoor airborne particulate sampler at the THI Site collected a sample between July 3, and July 11, 1985. No reactor related radioactivity was detected. Analysis showed I-131 and Cs-137 concentrations to be less than the lower limits of detectability.

5. REACTOR BUILDING ACTIVITIES

- Additional characterization of debris and inspection of the lower reactor head internals is scheduled during the week of July 14, 1985. Plenum removal on May 15, 1985, opened additional access ports to the lower head where 10 to 20 tons of debris was identified during a closed circuit television inspection in February 1985.

The characterization next week will include video inspections at two quadrants which had not been examined previously. The camera will be attached to an articulating arm having the capability to insert the camera into the 6 inch diameter flow holes in the lower flow distributor. Collection of debris and liquid samples will be attempted during the scheduled characterization. Mechanical probing of the lower head debris is included as part of the video inspection.

Defueling preparations will continue in parallel with the lower head characterization. Assembly of the defueling platform above the internals indexing fixture will commence next week. The trolley

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which will eventually transfer fuel canisters from the reactor vessel to the deep end of the refueling canal will be installed on the fuel transfer bridge.

6. AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES

- Installation of the DWCS continued. Partial DWCS turnover for processing RCS during early defueling is scheduled to be completed in late August.

7. NRC EVALUATIONS IN PROGRESS

- Defueling Water Cleanup System Technical Evaluation (including Revision 6)
- Technical Specification Change Requests numbers 46, 48, 49, and 50
- Recovery Operations Plan Change numbers 27, 29, 31, and 32
- Fuel Canister Technical Evaluation
- Fuel Handling Senior Reactor Operator Training Program
- Defueling Safety Evaluation
- Application for seismic exemption
- The NRC Vendor Programs Branch performed an inspection at the Nuclear Energy Services (NES) facility in Greensboro, North Carolina, where the fuel storage canisters, fuel storage racks, and fuel canister transfer shields are being fabricated. The inspection examined construction activities and included a review of the implementation of the quality assurance program at NES. The inspection findings are undergoing NRC management review and the final inspection report will be used in a few weeks.

8. PROJECTED SCHEDULE OF FUTURE EVENTS

- Start of Defueling: September 1985

9. PUBLIC MEETING

- The Advisory Panel for the Decontamination of Three Mile Island Unit 2 will meet on July 18, 1985, from 7:00 PM to 10:00 PM at the Lancaster Council Chambers, Public Safety Building, 201 North Duke Street, Lancaster, PA. The meeting will be open to the public.

At this meeting the Panel will receive a general update on the progress of the cleanup from General Public Utilities Nuclear Corporation, the licensee. The licensee will also provide a detailed discussion of the reactor pressure vessel defueling program. The staff of the U. S. Nuclear Regulatory Commission will provide the Panel with the results of a recent staff review of health effects studies conducted in the vicinity of TMI-2 since the

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March 28, 1979 accident. The Panel will also hold a planning session to identify and schedule future topics for Panel discussion.

Persons desiring the opportunity to speak before the Panel are asked to contact Mr. Thomas Smithgall at 717-291-1042 or write to him at 2122 Marietta Avenue, Lancaster, Pennsylvania 17603. Persons desiring to submit topics or questions for consideration by the Panel are asked to contact, in writing, Mayor Arthur Morris, 120 North Duke Street, Lancaster, Pennsylvania 17602.

**ORIGINAL SIGNED BY:**  
**William D. Travers**

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Deputy Program Director  
TMI Program Office

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Harold R. Denton  
Bernard J. Snyder

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